



Standard Practice for High-Resolution Gamma-Ray Spectrometry of Water¹

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1. Scope

1.1 This practice covers the measurement of gamma-ray emitting radionuclides in water by means of gamma-ray spectrometry. It is applicable to nuclides emitting gamma-rays with energies greater than 45 keV. For typical counting systems and sample types, activity levels of about 40 Bq are easily measured and sensitivities as low as 0.4 Bq are found for many nuclides. Count rates in excess of 2000 counts per second should be avoided because of electronic limitations. High count rate samples can be accommodated by dilution, by increasing the sample to detector distance, or by using digital signal processors.

1.2 This practice can be used for either quantitative or relative determinations. In relative counting work, the results may be expressed by comparison with an initial concentration of a given nuclide which is taken as 100 %. For quantitative measurements, the results may be expressed in terms of known nuclidic standards for the radionuclides known to be present. This practice can also be used just for the identification of gamma-ray emitting radionuclides in a sample without quantifying them. General information on radioactivity and the measurement of radiation has been published (1,2).² Information on specific application of gamma spectrometry is also available in the literature (3-5). See also the referenced ASTM Standards in 2.1 and the related material section at the end of this standard.

1.3 *This standard does not purport to address the safety concerns, if any, associated with its use. It is the responsibility of the user of this standard to establish appropriate safety and health practices and determine the applicability of regulatory limitation prior to use.*

¹ This practice is under the jurisdiction of ASTM Committee D19 on Water and is the direct responsibility of Subcommittee D19.04 on Methods of Radiochemical Analysis.

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² The boldface numbers in parentheses refer to a list of references at the end of this standard.

2. Referenced Documents

2.1 ASTM Standards:³

D1066 Practice for Sampling Steam

D1129 Terminology Relating to Water

D2777 Practice for Determination of Precision and Bias of Applicable Test Methods of Committee D19 on Water

D3370 Practices for Sampling Water from Closed Conduits

D3648 Practices for the Measurement of Radioactivity

D4448 Guide for Sampling Ground-Water Monitoring Wells

E181 Test Methods for Detector Calibration and Analysis of Radionuclides

3. Terminology

3.1 *Definitions*—For definitions of terms used in this practice, refer to Terminology D1129. For terms not defined in this practice or in Terminology D1129, reference may be made to other published glossaries.

4. Summary of Practice

4.1 Gamma ray spectra are measured with modular equipment consisting of a detector, high-voltage power supply, preamplifier, amplifier and analog-to-digital converter (or digital signal processor), multichannel analyzer, as well as a computer with display.

4.2 High-purity germanium (HPGe) detectors, p-type or n-type, are used for the analysis of complex gamma-ray spectra because of their excellent energy resolution. These germanium systems, however, are characterized by high cost and require cooling. Liquid nitrogen or electromechanical cooling, or both, can be used.

4.3 In a germanium semiconductor detector, gamma-ray photons produce electron-hole pairs. The charged pair is then collected by an applied electric field. A very stable low noise preamplifier is needed to amplify the pulses of electric charge

³ For referenced ASTM standards, visit the ASTM website, www.astm.org, or contact ASTM Customer Service at service@astm.org. For *Annual Book of ASTM Standards* volume information, refer to the standard's Document Summary page on the ASTM website.

resulting from gamma photon interactions. The output from the preamplifier is directly proportional to the energy deposited by the incident gamma-ray. These current pulses are fed into an amplifier of sufficient gain to produce voltage output pulses in the amplitude range from 0 to 10 V.

4.4 A multichannel pulse-height analyzer is used to determine the amplitude of each pulse originating in the detector, and accumulates in a memory the number of pulses in each amplitude band (or channel) in a given counting time. Computerized systems with stored programs and interface hardware can accomplish the same functions as hardwired multichannel analyzers. The primary advantages of the computerized system include the capability of programming the multi-channel analyzer functions and the ability to immediately perform data reduction calculations using the spectral data stored in the computer memory or mass storage device. For a 0 to 2-MeV spectrum, 4000 or more channels are typically needed in order to fully utilize a germanium detector's excellent energy resolution.

4.5 The distribution of the amplitudes (pulse heights) of the pulses can be separated into two principal components. One of these components has a nearly Gaussian distribution and is the result of total absorption of the gamma-ray energy in the detector. This peak is normally referred to as the full-energy peak or photopeak. The other component is a continuous one lower in energy than that of the photopeak. This continuous curve is referred to as the Compton continuum and is due to interactions wherein the gamma photons deposit only part of their energy in the detector. These two portions of the curve are shown in Fig. 1. Other peaks, such as escape peaks, backscattered gamma rays or X rays from shields, are often superimposed on the Compton continuum. Escape peaks will be present when gamma-rays with energies greater than 1.02 MeV are emitted from the sample. The positron formed in pair production is usually annihilated in the detector and one or both of the 511-keV annihilation quanta may escape from the detector without interaction. This condition will cause single or

double escape peaks at energies of 0.511 or 1.022 MeV less than the photopeak energy. In the plot of pulse height versus count rate, the size and location of the photopeak on the pulse height axis is proportional to the number and energy of the incident photons, and is the basis for the quantitative and qualitative application of the spectrometer. The Compton continuum serves no useful purpose in photopeak analysis and must be subtracted when peaks are analyzed.

4.6 If the analysis is being directed and monitored by an online computer program, the analysis period may be terminated by prerequisites incorporated in the program. If the analysis is being performed with a modern multichannel analyzer, analysis may be terminated when a preselected time or total counts in a region of interest or in a specified channel is reached. Visual inspection of a display of accumulated data can also be used as a criterion for manually terminating the analysis on either type of data acquisition systems.

4.7 Upon completion of the analysis, the spectral data are interpreted and reduced to include activity of Bq (disintegration per second) or related units suited to the particular application. At this time the spectral data may be inspected to identify the gamma-ray emitters present. This is accomplished by reading the channel number from the *x*-axis and converting to gamma-ray energy by multiplying by the appropriate keV/channel (system gain). In some systems the channel number or gamma-ray energy in keV can be displayed for any selected channel. Identification of nuclides may be aided by catalogs of gamma-ray spectra and other nuclear data tabulations (3,6-8).

4.7.1 Computer programs for data reduction have been used extensively although calculations for some applications can be performed effectively with the aid of a scientific calculator. Data reduction of spectra taken with germanium spectrometry systems is usually accomplished by integration of the photopeaks above a definable background (or baseline) and subsequent activity calculations using a library which includes data such as nuclide name, half-life, gamma-ray energies, and absolute gamma intensity.

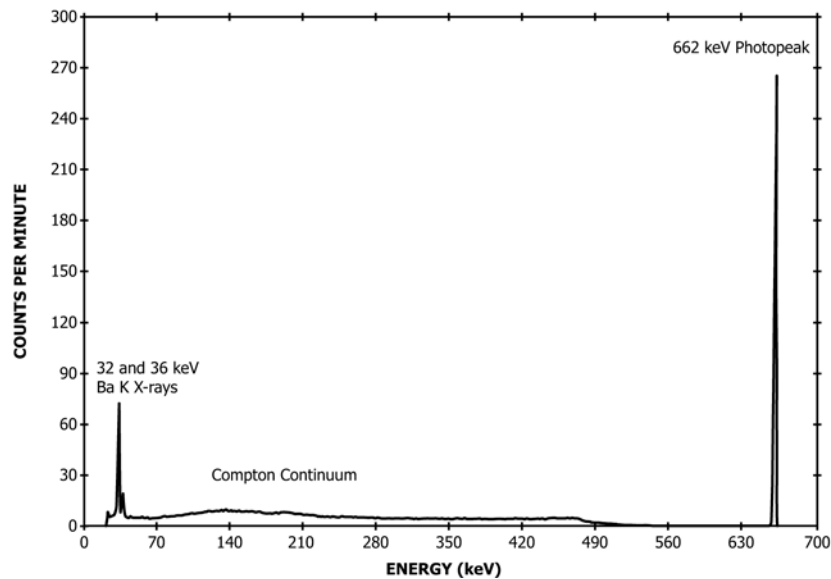


FIG. 1 Cesium-137 Spectrum

5. Significance and Use

5.1 Gamma-ray spectrometry is of use in identifying radionuclides and in making quantitative measurements. Use of a semiconductor detector is necessary for high-resolution measurements.

5.2 Variation of the physical geometry of the sample and its relationship with the detector will produce both qualitative and quantitative variations in the gamma-ray spectrum. To adequately account for these geometry effects, calibrations are designed to duplicate all conditions including source-to-detector distance, sample shape and size, and sample matrix encountered when samples are measured.

5.3 Since some spectrometry systems are calibrated at many discrete distances from the detector, a wide range of activity levels can be measured on the same detector. For high-level samples, extremely low-efficiency geometries may be used. Quantitative measurements can be made accurately and precisely when high activity level samples are placed at distances of 10 cm or more from the detector.

5.4 Electronic problems, such as erroneous deadtime correction, loss of resolution, and random summing, may be avoided by keeping the gross count rate below 2000 counts per second (s^{-1}) and also keeping the deadtime of the analyzer below 5 %. Total counting time is governed by the radioactivity of the sample, the detector to source distance and the acceptable Poisson counting uncertainty.

6. Interferences

6.1 In complex mixtures of gamma-ray emitters, the degree of interference of one nuclide in the determination of another is governed by several factors. If the gamma-ray emission rates from different radionuclides are similar, interference will occur when the photopeaks are not completely resolved and overlap. If the nuclides are present in the mixture in unequal portions radiometrically, and if nuclides of higher gamma-ray energies are predominant, there are serious interferences with the interpretation of minor, less energetic gamma-ray photopeaks. The complexity of the analysis method is due to the resolution of these interferences and, thus, one of the main reasons for computerized systems.

6.2 Cascade summing may occur when nuclides that decay by a gamma-ray cascade are analyzed. Cobalt-60 is an example; 1172 and 1333-keV gamma rays from the same decay may enter the detector to produce a sum peak at 2505 keV and cause the loss of counts from the other two peaks. Cascade summing may be reduced by increasing the source to detector distance. Summing is more significant if a well-type detector is used.

6.3 Random summing is a function of counting rate and occurs in all measurements. The random summing rate is proportional to the total count squared and the resolving time of the detector. For most systems random summing losses can be held to less than 1 % by limiting the total counting rate to 2000 counts per second (s^{-1}). Refer to Test Methods E181 for more information.

6.4 The density of the sample is another factor that can effect quantitative results. Errors from this source can be

avoided by preparing the standards for calibration in solutions or other matrices with a density comparable to the sample being analyzed.

7. Apparatus

7.1 *Gamma Ray Spectrometer*, consisting of the following components:

7.1.1 *Detector Assembly*:

7.1.1.1 *Germanium Detector*—The detector may have a volume of about 50 to 150 cm^3 , with a full width at one-half the peak maximum (FWHM) less than 2.2 keV at 1332 keV, certified by the manufacturer. A charge-sensitive preamplifier using low noise field effect transistors should be an integral part of the detector assembly. A convenient support should be provided for samples of the desired form.

7.1.1.2 *Shield*—The detector assembly may be surrounded by an external radiation shield made of a dense metal, equivalent to 102 mm of lead in gamma-ray attenuation capability. It is desirable that the inner walls of the shield be at least 127 mm distant from the detector surfaces to reduce backscatter. If the shield is made of lead or a lead liner, the shield may have a graded inner shield of 1.6 mm of cadmium or tin lined with 0.4 mm of copper, to attenuate the 88-keV Pb X-rays. The shield should have a door or port for inserting and removing samples.

7.1.1.3 *High Voltage Power/Bias Supply*—The bias supply required for germanium detectors usually provides a voltage up to 5000 V and up to 100 μA . The power supply should be regulated to 0.1 % with a ripple of not more than 0.01 %. Line noise caused by other equipment should be removed with rf filters and additional regulators.

7.1.1.4 *Amplifier*—An amplifier compatible with the preamplifier and with the pulse-height analyzer shall be provided.

7.1.2 *Data Acquisition and Storage Equipment*:

7.1.2.1 *Data Acquisitions*—A multichannel pulse-height analyzer (MCA) or stand-alone analog-to-digital-converter (ADC) under software control of a separate computer, performs many functions required for gamma-ray spectrometry. An MCA or computer collects the data, provides a visual display, and outputs final results or raw data for later analysis. The four major components of an MCA are the ADC, the memory, control, and input/output. More recently, digital signal processors (DSP) can directly amplify and digitize signals from the preamplifier, replacing individual amplifier and ADC components. The ADC digitizes the analog pulses from the amplifier. These pulses represent energy. The digital result is used by the MCA to select a memory location (channel number) which is used to store the number of events which have occurred with that energy. Simple data analysis and control of the MCA is accomplished with microprocessors. These processors control the input/output, channel summing over set regions of interest, and system energy calibration to name a few examples.

7.1.2.2 *Data Storage*—Because of the use of microprocessors modern MCAs provide a wide range of input and output (I/O) capabilities.

8. Sampling

8.1 Collect the sample in accordance with Practice **D1066**, Practices **D3370**, Guide **D4448**, or other documented procedures.

8.2 Preserve the sample in a radioactively homogeneous state. A sample can be made radioactively homogeneous by the addition of a reagent in which the radionuclides or compounds of the radionuclides present would be soluble in large concentrations. Addition of acids, complexing agents, or stable, chemically similar carriers may be used to obtain homogeneity. Consideration of the chemical nature of the radionuclides and compounds present and the subsequent chemistry of the method shall indicate the action to be taken.

9. Test Specimens

9.1 *Containment*—Sample mounts and sample-counting containers must have a convenient and reproducible geometry. Considerations include commercial availability, ease of use and disposal, and the containment of radioactivity for protection of the working environment and personnel from contamination. The evaporation of liquid samples to dryness is not necessary and liquid samples up to several litres may be used. However, samples that have been evaporated to dryness for gross beta counting can also be gamma counted. Massive samples may cause significant self-absorption of low-energy gammas and degrade the higher-energy gammas. Therefore, it is important to calibrate the detector with standards of the same geometry and density. A beta absorber consisting of about 6 mm of aluminum, beryllium, or plastic may be used for samples that have a significant beta activity and high beta energies.

10. Calibration and Standardization

10.1 *Overview*—The instrumentation and detector should be put into operation according to the manufacturer's instructions. Initial set-up includes all electronic adjustment to provide constant operating conditions consistent with the application and expected duration of the calibrations. The analog-to-digital converter gain and threshold, amplifier gain and zero-level, and detector high voltage, or bias must be adjusted to yield an optimum energy calibration, usually 1 keV, or less, per channel (0.5 keV/channel is recommended). Modern commercial equipment is capable of linearity to the extent that the energy may be interpreted by the operator directly to the nearest 0.5 keV simply by reading the channel number of the highest channel in a peak and using the energy calibration data to calculate the energy of the peak. The energy calibration is usually accomplished with radioactive sources covering the entire range of interest. Subsequent efficiency calibrations and source analyses are performed with the same gain settings and the same high voltage setting. Efficiency calibrations are obtained by placing an appropriate volume of a radionuclide standard solution containing 1 to 1000 kBq in a container and placing the container on the detector or in the well of the detector.

10.2 Procedure:

10.2.1 Preparation of Apparatus:

10.2.1.1 Follow the manufacturer's instructions, limitations, and cautions for the setup and the preliminary testing for all of the spectrometry equipment to be used in the analysis. This equipment would include, as applicable, detector, power supplies, preamplifiers, amplifiers, multichannel analyzers, and computing systems.

10.2.1.2 Place an appropriate volume of a standard or a mixed standard of radionuclides in a sealed container and place the container at a desirable and reproducible source-to-detector distance. Section 6 provides information on cascade and random summing interferences that should be considered when establishing a source-to-detector distance. The solution should provide about 100 counts per second (s^{-1}) in the peaks of interest and be made up of standard sources traceable to a nationally certified laboratory. In all radionuclide measurements, the volumes, shape, physical and chemical characteristics of the samples, standards and their containers must be as equivalent as practicable for the most accurate results. If precipitates or residues are to be analyzed, then the standards must be evaporated on the same type of mount as the sample.

10.3 Energy Calibration:

10.3.1 The energy calibration (channel number of the multichannel analyzer versus the gamma-ray energy) of the detector system is accomplished at a fixed gain using standards containing known radionuclides. The standards should be in sealed containers and should emit at least four different gamma-ray energies covering the range of interest, usually 50 to 2000 keV in order to determine coefficients for the energy fitting function. Some commercially available nuclides suitable for energy calibration are shown in **Table 1**.

10.3.1.1 Mixed gamma-ray standards for energy and efficiency calibration are also available (see **Fig. 2** for an example). These standards can be obtained in solid form in a user-supplied container.

10.3.2 A multichannel analyzer should be calibrated to cover the energy range of interest. If the range is from 50 to 2000 keV, the gain of the system shall be adjusted until the ^{137}Cs photopeak, 662 keV, is about one-third full scale. Leaving the gain constant, locate at least three other photopeaks of different energies, covering the same range. Determine and record the multichannel analyzer channel number corresponding to the maximum count rate for each of the four gamma energies. Plot the gamma energy versus the channel

TABLE 1 Energy Calibration Nuclides

Nuclide	Approximate energy, keV
lead-210	46.5
americium-241	59.5
cadmium-109	88
cerium-141	145
chromium-51	320
cesium-137	662
manganese-54	835
sodium-22	511 and 1275
yttrium-88	898 and 1836
cobalt-60	1173 and 1332
europium-152	122, 344, 779, 964, 1112, and 1408
radium-226 in equilibrium	186, 352, 609, 1120, and 1765

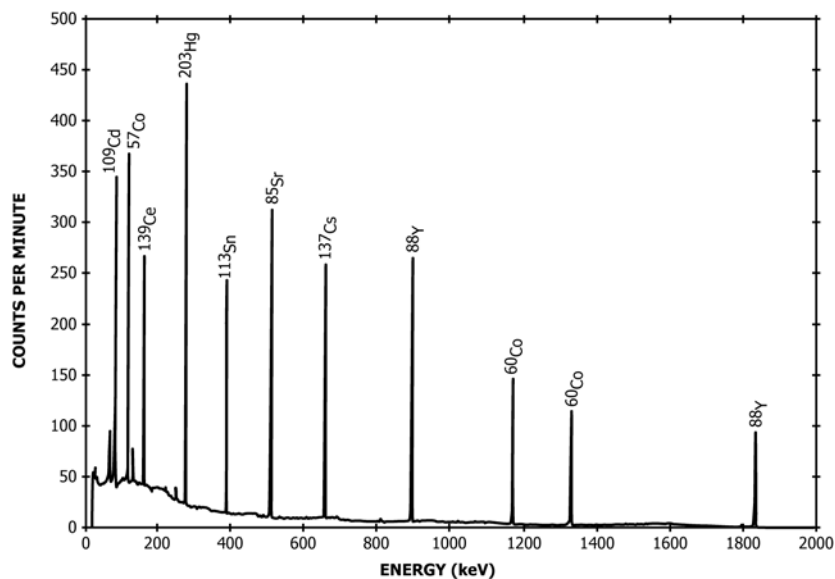


FIG. 2 Mixed Gamma-Ray Calibration Spectrum

number for each of the four gamma-ray energies. A linear or quadratic relationship will be observed if the equipment is operating properly. Samples should not be analyzed if there is not such a relationship. Calculate the coefficients of the fitting function. If the spectrometry system is computer controlled, follow the appropriate manufacturer input instructions for the determination of the coefficients. The energy calibration should be verified at a predetermined interval. If the coefficients are essentially unchanged, the energy calibration data remain valid. If an appreciable change in the coefficients is evident, the energy calibration procedure must be rerun.

10.4 Photon Detection Efficiency Calibration:

10.4.1 Accumulate an energy spectrum using sealed, calibrated radioactivity standards in a desired and reproducible counting geometry (see 10.2.1.2). At least 10 000 net counts (total counts minus the Compton continuum and ambient background) shall be accumulated in each full-energy gamma-ray peak of interest. Compare the live time of the count to the half-life of the radionuclide of interest. If the live time is greater than 5 % of the half-life, a correction factor must be applied for decay during the count.

10.4.2 Correct the radioactivity standard source gamma-ray emission rate for the decay from the time of standardization to the time at which the count rate is measured.

10.4.3 Calculate the full-energy peak efficiency, ϵ_f , as follows:

$$\epsilon_f = R_{\text{net}}/R_\gamma \quad (1)$$

where:

- ϵ_f = full-energy peak efficiency (counts per gamma-ray emitted),
- R_{net} = net gamma-ray count in the full-energy peak of interest (counts per second), and
- R_γ = gamma-ray emission rate (gamma rays per second).
If the standard source is calibrated as to activity, the gamma-ray emission rate is given by:

$$R_\gamma = A \times I \quad (2)$$

where:

- A = activity in becquerels (Bq), and
- I = absolute gamma intensity for the specific gamma-ray emission.

10.4.4 Many modern spectrometry systems are computerized and the determination of the gamma-ray efficiencies are determined automatically at the end of an appropriate counting interval. Refer to the manufacturer’s instructions for specific output requirements.

10.4.5 Plot the values for the full-energy peak efficiency (as determined in 10.4.3) versus gamma-ray energy. The plot will allow the determination of efficiencies at energies for which standards are not available and to show that the algorithms used in computerized systems are providing valid efficiency calibrations. A typical plot is shown in Fig. 3.

10.4.6 Once the efficiencies have been determined, it is unnecessary to recalculate them unless there is a change in resolution, geometry, or system configuration.

11. Sample Measurements

11.1 After the spectrometer system has been set up and the energy and efficiency calibrations performed, unknown specimens can be measured.

11.2 Following the general concepts of quantitative analytical chemistry, transfer the sample to the specimen container (9.1) and position it in the same manner as was done during system calibration (Section 10).

11.3 Measure the sample for a period of time long enough to acquire a gamma-ray spectrum which will meet the minimum acceptable counting uncertainty.

12. Calculation

12.1 In many experiments, the background may not affect the results but is still monitored to ensure the integrity of the system. Again, the practice presented here is not the only type

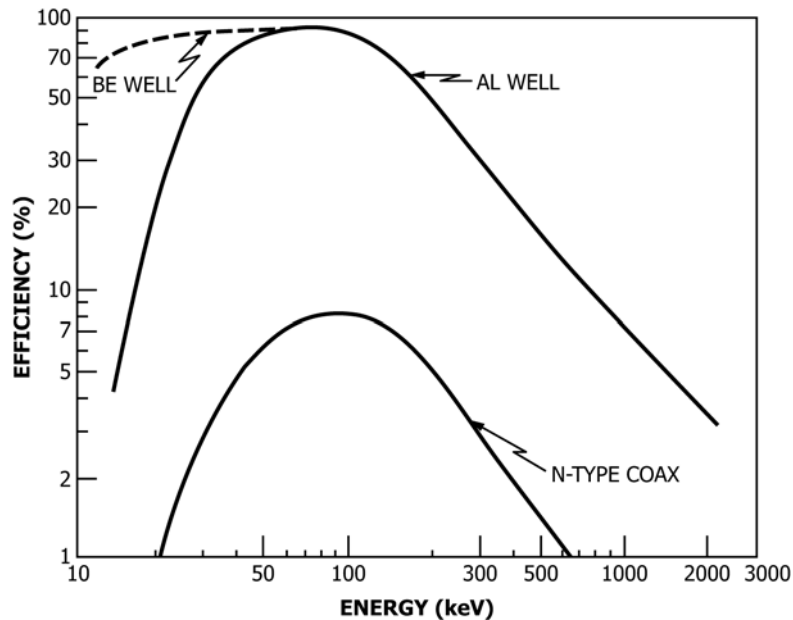


FIG. 3 Efficiency Calibration Plot

but is conducive to available computational hardware and should be used to verify the validity of commercial software.

12.2 The underlying aim of this practice is to subtract the continuum or baseline from the spectral data where it underlies a photopeak of interest. For interactive calculations, the choice of the baseline level may be straight-forward. The simplest way, using a plot of the spectral data, is to draw a straight line, using judgement and experience, that best describes the baseline. Then the baseline data can be read directly from the plot and subtracted. “Stand-alone” computer programs have accomplished this but are not presented here.

12.3 Photopeaks lying on a sloping baseline or one with curvature will be analyzed, independent of method, with increased uncertainty. Use of data from these peaks should be limited to those cases where there is no other alternative. Photopeaks that overlap with each other will also increase the uncertainty of the final result. In the case where use of overlapping peaks cannot be avoided, the experimenter may estimate the areas by assuming that the ratio of the peak areas is equal to the ratio of the peak heights. Various software-based approaches available will separate overlapping peaks with varying degrees of success.

12.4 In order to determine nuclide concentrations, the photopeak areas corrected for background and interferences are divided by the count time and efficiency for the energy of the gamma ray being calculated to give gammas per second (s^{-1}) for the peak of interest. If, as is the case for some nuclides, the absolute gamma intensity is not accurately known and a direct calibration was made with the same nuclide, the absolute gamma intensity and efficiency will be one number that converts counts per second (s^{-1}) to disintegrations per second for the nuclide and photopeak of interest. If not, the gammas per second are converted to disintegrations per second by dividing the gammas per second by the gammas per disintegration, for the nuclide and photopeak of interest (6-8).

The results are then corrected for sample size or decay, or both, as demanded by the application.

12.5 Results are expressed in becquerels per litre (Bq/L). For the simple cases of radionuclides where there are no interfering photopeaks from other radionuclides in the sample or in the background, calculate the nuclide activity concentration, AC , by the following equation:

$$AC = \frac{R_{net}}{(\epsilon \times V \times I \times DF)} \quad (3)$$

where:

- R_{net} = net count rate (s^{-1}),
- V = test specimen volume, L,
- ϵ = detector efficiency factor,
- I = absolute gamma intensity, and
- DF = radionuclide decay correction factor (correction for radioactive decay).

12.6 The combined standard uncertainty of the nuclide activity concentration $u_c(AC)$ is calculated as:

$$u_c(AC) = \quad (4)$$

$$\sqrt{\frac{u^2(R_{net})}{\epsilon^2 \times V^2 \times I^2 \times DF^2} + AC^2 \times \left(\frac{u^2(\epsilon)}{\epsilon^2} + \frac{u^2(V)}{V^2} + \frac{u^2(I)}{I^2} + \frac{u^2(DF)}{DF^2} \right)}$$

where:

- $u(R_{net})$ = standard uncertainty of the net counting rate,
- $u(\epsilon)/\epsilon$ = relative standard uncertainty of the detector efficiency,
- $u(V)/V$ = relative standard uncertainty of the sample volume measurement,
- $u(I)/I$ = relative standard uncertainty of the of the absolute gamma intensity, and
- $u(DF)/DF$ = relative standard uncertainty of the decay factor.

12.7 The net count rate R_{net} and counting uncertainty, $u(R_{net})$, are defined as:

$$R_{\text{net}} = R_s - \left(R_b \times \frac{n_p}{n_b} \right) = \frac{C_s - (C_b \times n_p/n_b)}{t_s} \quad (5)$$

$$u(R_{\text{net}}) = \frac{\sqrt{C_s + (C_b \times n_p^2/n_b^2)}}{t_s} \quad (6)$$

where:

- R_s = photopeak count rate (s^{-1}),
- R_b = baseline background count rate (s^{-1}),
- C_s = photopeak counts,
- C_b = baseline counts,
- t_s = counting time of sample (s),
- n_p = number of channels in the photopeak, and
- n_b = number of channels used in the baseline subtraction.

12.8 The *a priori* minimum detectable concentration (MDC), assuming a detection decision based on a single pre-selected gamma-ray photopeak, is calculated using the equation:

$$MDC = \frac{1}{k} \times \left(\frac{2.71}{t_s} + 3.29 \sqrt{\frac{R_b \times (F + F^2)}{t_s}} \right) \quad (7)$$

where:

- $F = n_{cp}/n_{cb}$, and
- $k = \varepsilon \times V \times I \times DF$.

12.9 A more detailed discussion on the minimum detectable concentration concept can be found in Practices [D3648](#).

13. Precision and Bias

13.1 This practice is utilized for the measurement of a wide range of water sample types which may contain only a few (one to five) gamma-ray emitting nuclides or contain a large number (or a very complex mixture) of nuclides giving rise to complex spectra having interfering photon peaks. Furthermore, the concentration ranges and half-lives of individual nuclides present in a given sample may vary over several orders of magnitude. Therefore, developing a round robin test plan in accordance with Practice [D2777](#) would not provide meaningful estimates of precision and bias.

13.2 Data published by the U.S. Department of Energy—Environmental Measurements Laboratory (EML) from their contractor Quality Assessment Program from 1982 to 1990 have been used to provide estimates of precision and bias. The U.S. Department of Energy—Environmental Measurements Laboratory contractor Quality Assessment Program provided about 60 laboratories with spiked samples twice a year. In almost all cases the data reported to EML were based upon a single measurement. Data published by this program from 1982 to 1990 are presented in [Table 2](#).

TABLE 2 Estimation of Precision and Bias (See 13.1)

Nuclide	Concentration Range, Bq/L	Number of Studies	Number of Observations	Average Bias, %	Average Precision, S_p , %
Mn-54	80 to 1600	13	434	+3	4
Co-57	6 to 195	6	221	+1	6
Co-60	30 to 185	12	403	-1	5
Cs-134	40 to 430	5	195	-5	9
Cs-137	65 to 200	15	485	+1	4
Ce-144	70 to 460	6	136	+7	7

14. Quality Control

14.1 Before this practice is utilized for the analysis of samples, a quality control or tolerance chart shall be established to ensure that the spectrometry system is operating within prescribed limits. The quality control or tolerance chart shall be established at the time the spectrometry system is calibrated.

14.2 Tolerance or statistical control charts are used to assure that an instrument or spectrometry system is operating to within prespecified limits of the initial calibration. Repetitive measurements of a quality control source are taken to develop the tolerance or statistical control chart. An instrument QC check then is performed on a daily or prior to use basis, whichever is less often, to ensure proper operation. The result of the QC check shall be tabulated or plotted on the control or tolerance chart and evaluated according to Practices [D3648](#). Refer to Practices [D3648](#) for the preparation of a tolerance of statistical control chart.

14.3 The QC source for the detector efficiency check should contain several gamma-ray energies of sufficiently long half-life to eliminate significant decay corrections. The selection of the radionuclide should provide a calibration check at three widely spaced energies, preferably one for the lower energies (^{241}Am – 59.5 keV), one at the intermediate energies (^{137}Cs – 661.6 keV), and one at the higher energies (^{60}Co – 1332.5 keV). This same source may be used to determine the energy calibration of the spectrometry system. Knowledge of the true disintegration rate of the radionuclides in the QC source is not essential as long as the source is decay corrected properly. Quality control sources are available commercially.

14.4 The full-width-half-maximum (FWHM) resolution of the detector should be evaluated. Maintaining a constant photopeak resolution is extremely important for some computer software algorithms that use the detector resolution in the spectral unfolding routines. The gamma-ray photopeaks used for the detector efficiency check can also be used for the resolution check. Under routine operation and normal amplifier and ADC settings, the keV/channel gain may be insufficient to verify the manufacturer's detector resolution specification or slight changes to the manufacturer's specifications; however, it is good practice to perform a resolution check of the system to identify gross changes in the resolution of the detector at energies that bound the applicable energy range. The increase in resolution that can be tolerated should be determined for the application under consideration or for the spectral unfolding computer software algorithm. The measured FWHM resolution of the selected photopeaks should be trended and corrective actions taken when an intolerable condition becomes evident.

14.5 Evaluate the counting system background periodically. The background data shall be maintained in a logbook or plotted on a trend chart.

14.6 Accuracy can be assessed by the analysis of NIST or other national standards laboratories traceable spiked samples with known quantities of radioactivity. Accuracy should not be assessed using solutions derived from the same standards that were used for the detector calibration. Accuracy also can be

evaluated by the participation in one of various commercial and government quality assurance programs that provide performance evaluation samples on a frequent basis.

15. Keywords

15.1 gamma-ray spectrometry; germanium detectors; high-resolution gamma-ray spectrometry; HPGe; photon spectrometry; water

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RELATED MATERIAL

ANSI N42.14, Standard for Calibration and Use of Germanium Spectrometers for the Measurement of Gamma-Ray Emission Rates of Radionuclides.⁴

Multi-Agency Radiological Laboratory Analytical Protocols Manual, July 2004, NUREG-1576, EPA 402-B-04-001A, NTIS PB2004-105421.

⁴ Available from American National Standards Institute (ANSI), 25 W. 43rd St., 4th Floor, New York, NY 10036, <http://www.ansi.org>.

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